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Energy Systems Group

# SUPPORTING DOCUMENT

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<table border="1"><thead><tr><th>*</th><th>NAME</th><th>MAIL ADDR</th></tr></thead><tbody><tr><td>*</td><td>F. E. Begley</td><td>NB13</td></tr><tr><td>*</td><td>C. C. Conners (10)</td><td>NB02</td></tr><tr><td>*</td><td>M. E. Remley</td><td>NB13</td></tr><tr><td>*</td><td>D. L. Speed</td><td>NB13</td></tr><tr><td>*</td><td>R. J. Tuttle (3)</td><td>NB13</td></tr><tr><td>*</td><td>J. D. Moore</td><td>NB13</td></tr><tr><td>*</td><td>R. W. Hartzler</td><td>201F</td></tr><tr><td>*</td><td>R. K. Holbrook</td><td>LA07</td></tr></tbody></table>			*	NAME	MAIL ADDR	*	F. E. Begley	NB13	*	C. C. Conners (10)	NB02	*	M. E. Remley	NB13	*	D. L. Speed	NB13	*	R. J. Tuttle (3)	NB13	*	J. D. Moore	NB13	*	R. W. Hartzler	201F	*	R. K. Holbrook	LA07	Following cleanup of any previously detected radio-activity exceeding specified limits, a radiation survey was performed in the parts of Building 004 used on the ATR Fuel Fabrication Program. The results of this survey show that these regions meet the criteria established by NRC for release for unrestricted use.		
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## I. INTRODUCTION

A portion of Building 004 at the Headquarters site was used on the ATR Fuel Fabrication Program for analysis of reactor fuel ( $UA1_x$  powder) using enriched uranium. This work has most recently been conducted under the NRC Special Nuclear Materials License No. SNM-21.<sup>(1)</sup> It was decided to decontaminate this portion of Building 004 at Headquarters, and eliminate it from this license.\*

Conditions 22 and 46 of the license impose Annex C (attached here as Appendix B) as a requirement for decontamination of facilities and equipment for release for unrestricted use. The requirements of Annex C have been followed.

TABLE I-1  
SUMMARY OF SURVEY RESULTS  
REGIONS IA and IB

Measurement	Number of Locations	Average Value (dpm/100 cm <sup>2</sup> )	Maximum Value (dpm/100 cm <sup>2</sup> )	Limit
Average alpha	155	18.74	344	5,000
Removable alpha	155	1.66	12	1,000
Average beta	155	492	2274	5,000
Removable beta	155	5.51	84	1,000

In all cases, the maximum value is well below the limit. The results summarized in this table confirm that the area is acceptable for release for unrestricted use.

\*The working document which describes this activity is ESG N065ACR630004, "Building 004 Analytical Radiochemistry Labs Decontamination and Decommissioning Activity Requirements," dated December 1, 1982.



## II. IDENTIFICATION OF PREMISES

The premises to be released consist of part of Building 004 at the Headquarters (or De Soto) site. This site is shown in Figure 1. It is located at 8900 De Soto Avenue in Canoga Park, California.

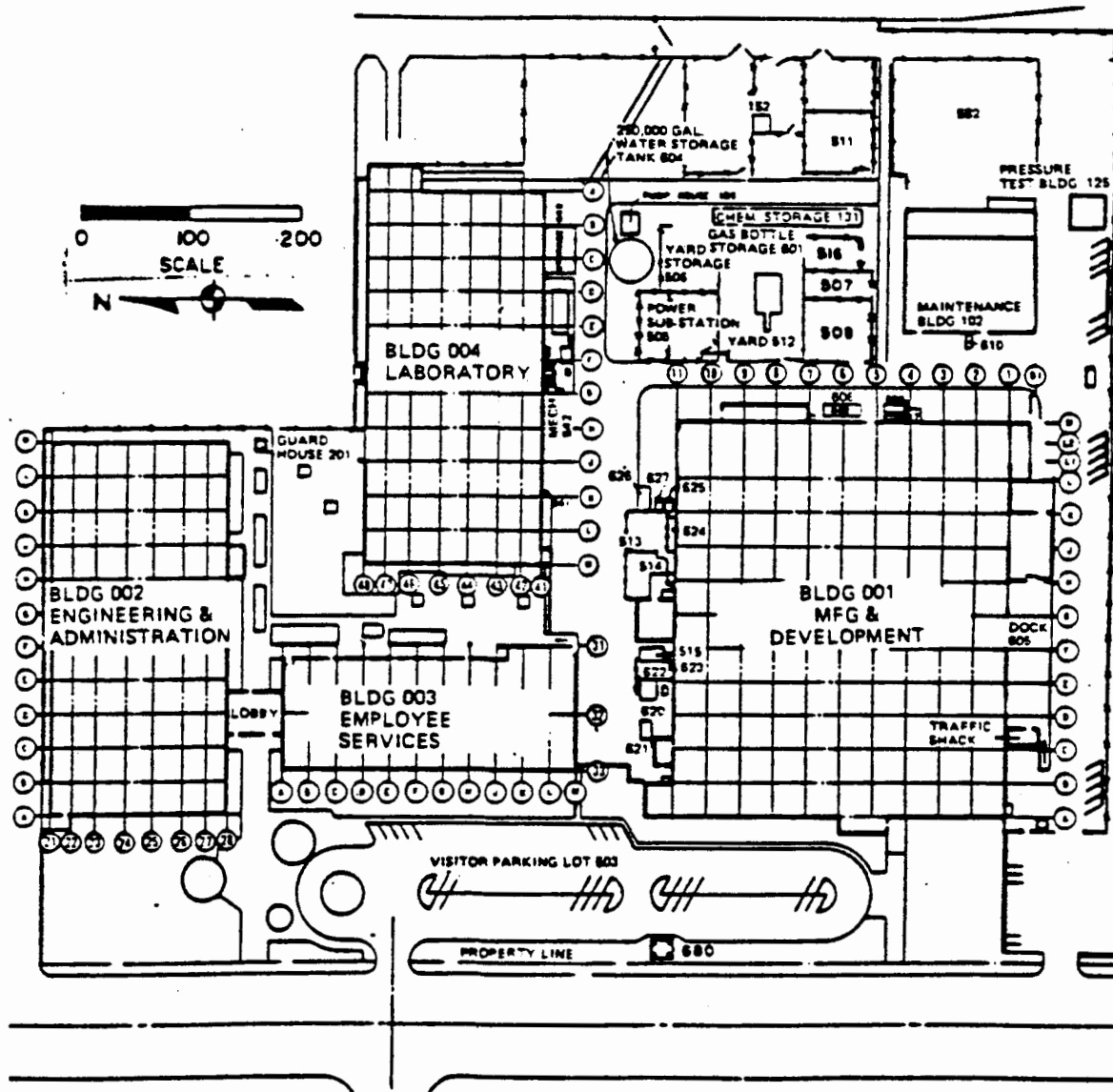


Figure 1. De Soto Avenue (Headquarters) Site in Canoga Park



Figure 2 shows the main radiochemistry laboratory located on the second floor of Building 004. Two additional rooms on the first floor, used for spectroscopy analysis, were also decontaminated in this effort.

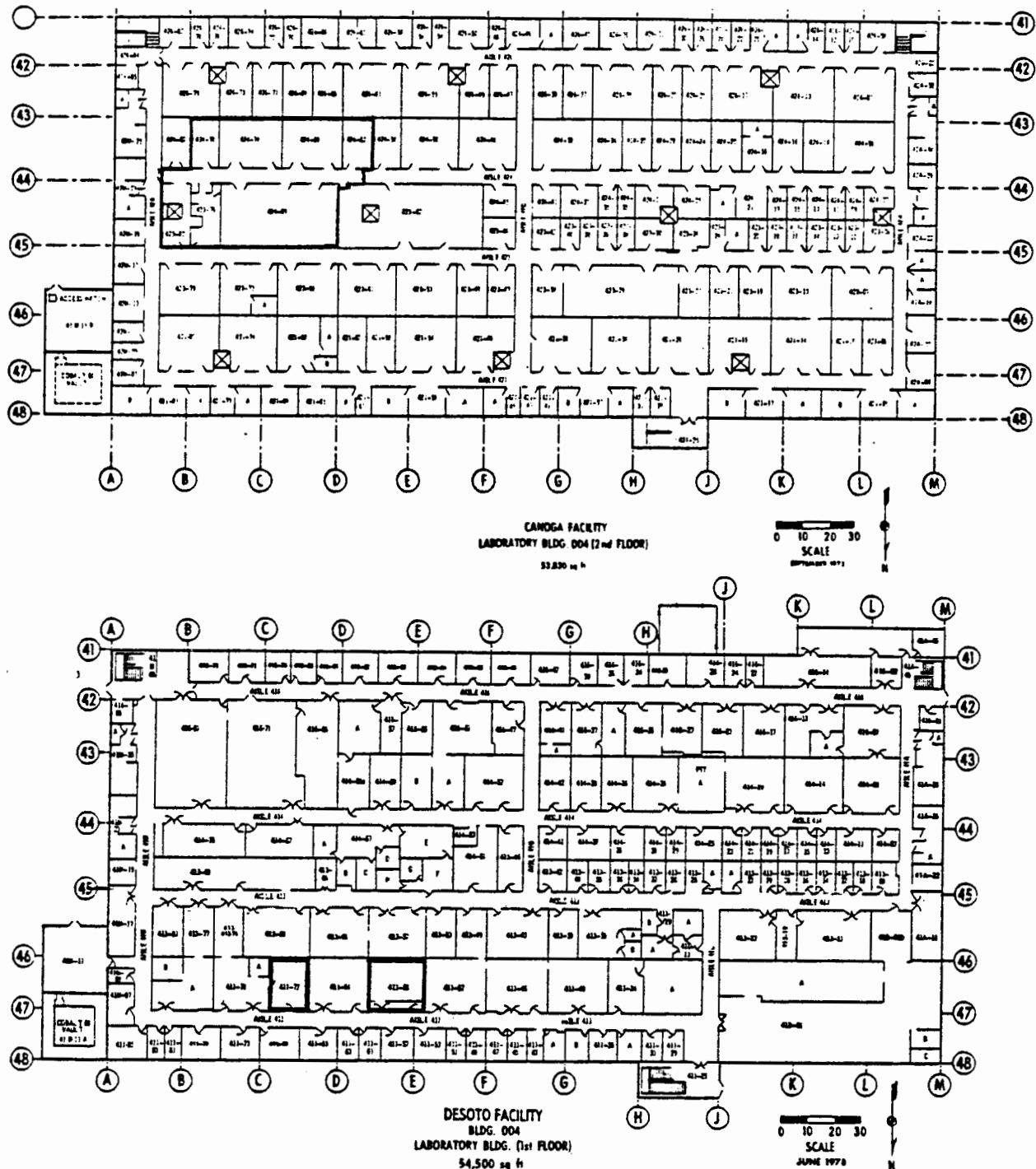


Figure 2. Building 004 Radiochemistry Laboratory



### III. DECONTAMINATION EFFORTS

This area was used for the analysis of ATR reactor fuel ( $UA1_x$  powder) using a uranium enrichment of 93%. This work began in 1959. Uranium analyses in these areas had ended by 1983 and, as the projects ended and as work areas were relocated, the readily accessible areas were generally cleaned to acceptable levels.

The comprehensive decontamination effort was commenced per ref. Act. Spec. This effort included the removal of all laboratory equipment and facilities (hoods, benches, cabinets, etc.), removal of radioactive exhaust facilities and drain lines, and removal of floor tile. Small amounts of residual activity were detected on the concrete floor surface and were removed by scabbling. All materials leaving the area were monitored for contamination and disposed of by land burial when in excess of regulatory guidelines for unrestricted use.



#### IV. SURVEY SCOPE AND PROCEDURES

##### A. SURVEY SCOPE

A sampling inspection plan using variables has been used to demonstrate that the residual contamination in the area is below the following limits:

Total average over $1 \text{ m}^2$	5,000 dpm/100 $\text{cm}^2$
Total maximum over 100 $\text{cm}^2$	15,000 dpm/100 $\text{cm}^2$
Removable	1,000 dpm/100 $\text{cm}^2$

The sampling inspection plan that was used is based upon a uniform 3-m (10-ft) square grid superimposed on the area. A 3-m-square grid has been adopted to be consistent with NRC and State of California guidance. The actual grid in each room was benchmarked in the northwest corner of the room. An identical grid was reflected onto the ceiling. A similar grid structure was also applied to the walls, benchmarked in the upper left corner of the walls. Each survey area has been identified with codes indicating the surface (F = floor; C = ceiling; N, E, S, W = north, east, south, and west walls, respectively) and a two-figure Cartesian coordinate showing the distance in meters from a local benchmark.

Within each square defined by the grid lines, a single  $1\text{-m}^2$  area was surveyed. Each area was outlined by felt marker or paint, with its coordinates marked within or beside the  $1\text{-m}^2$  area. The location of this  $1\text{-m}^2$  area was left to the surveyor's judgment: it was to be the area that, in his judgment, was most likely to have retained the most residual contamination of any similar area within the grid square. The surveyor was instructed to do this conscientiously to assure that any significant residual contamination would be detected before a report of acceptability was made to a regulatory agency. The use of a predetermined grid with discretion for the exact location provides a biased-uniform survey; selection of one  $1\text{-m}^2$  area out of the nine within each grid square provides an 11% sampling of the surface.



Sampling inspection consists of a sampling plan for selection of items to be tested--in this case, locations to be measured for radioactivity, and the method of analysis. The sampling plan used for this phase was to inspect one 1-m<sup>2</sup> area out of every other 3-m grid square throughout the regions.

This 11% inspection (compared to 10% as recommended by the State of California) was used for these areas.

The 1-m<sup>2</sup> area chosen by the procedure described above is first measured for total alpha and beta activity and then for removable activity.

## B. PROCEDURES

The following procedures were used in performing this survey.

### 1. Average Contamination Measurement

- 1) Identify 1-m<sup>2</sup> area to be measured.
- 2) With a portable scaler (Technical Associates FS-8 or equivalent) set for 5-min count time, use an alpha probe (Ludlum Model 43-1 or equivalent) or a beta probe (Ludlum Model 44-9 or Associates Model P-11 or equivalent) and uniformly scan the area. (Watch for and note any "hot spots" where the radioactivity may exceed the average limit. These are to be resurveyed later.)
- 3) Record the location and total count.
- 4) The total count is converted to dpm/100 cm<sup>2</sup> total surface activity by:

$$SA_T = \left( \frac{C - B}{5} \right) E \left( \frac{100}{A} \right)$$





where

$SA_T$  = total surface activity in dpm/100 cm<sup>2</sup>

C = total count in 5 min

5 = count time, min

B = background count in 5 min (generally 0-5 for alpha and about 200-220 for beta)

E = efficiency factor, dpm/cpm (generally 4 for alpha and 7 for beta)

100 = 100 cm<sup>2</sup> standard area

A = probe sensitive area (69 cm<sup>2</sup> for Ludlum Model 43-1 alpha scintillator; 20 cm<sup>2</sup> for Ludlum Model 44-9 and Technical Associates Model P-11 pancake G-M).

(Note that the analysis is done using counts rather than count rates.)

## 2. Maximum Contamination Measurement

- 1) Return to any area identified as having a "hot spot."
- 2) Repeat the uniform scan of only the hot spot area, covering approximately 100 cm<sup>2</sup> with the probe.
- 3) Record the location and total count as a "hot spot" measurement.
- 4) The total count is converted to dpm/100 cm<sup>2</sup> as shown above.

## 3. Removable Contamination Measurement

- 1) Identify 1-m<sup>2</sup> area to be measured.
- 2) Using a Whatman 540 filter paper (2.4 cm diameter), wipe a "Z" or "S" pattern, with legs approximately 6 in. long, so as to sample removable contamination from an area of approximately 100 cm<sup>2</sup>.
- 3) Place smear paper in file card "book" until ready for counting.
- 4) Count radioactivity using gas-flow proportional counter (NMC Model ACS-77 or equivalent) for 5 min.



- 5) Record the location and both the total alpha count and the total beta count.
- 6) The total counts are converted to dpm/100 cm<sup>2</sup> removable surface activity by:

$$SA_R = \left( \frac{C - B}{5} \right) E$$

where the appropriate alpha and beta backgrounds and efficiency factors are used. Backgrounds are typically 1-3 counts for alpha and 120-150 counts for beta. Efficiency factors are about 4 dpm/cpm for alpha and beta.



## V. SURVEY RESULTS

The survey of this area was conducted using the aforementioned standard survey plan. No reduction in sampling was applied to this area. The results of the survey are listed in Appendix A. A summary of the survey results appears below in Table V-1. Due to project scheduling considerations, and the extremely low levels of residual contamination measured in this survey, it was decided that statistical data analysis was not required for this limited area.

TABLE V-1  
SUMMARY OF SURVEY RESULTS  
REGIONS IA and IB

Measurement	Number of Locations	Average Value (dpm/100 cm <sup>2</sup> )	Maximum Value (dpm/100 cm <sup>2</sup> )	Limit
Average alpha	155	18.74	344	5,000
Removable alpha	155	1.66	12	1,000
Average beta	155	492	2274	5,000
Removable beta	155	5.51	84	1,000

In all cases, the maximum value is well below the limit. The results summarized in this table confirm that the area is acceptable for release for unrestricted use.



## VI. CONCLUSIONS

An appropriate survey has been conducted throughout the area to be released. All remaining measured values of residual radioactivity are below the acceptance limit. The results of this survey show essentially no residual contamination and demonstrate a negligible risk of there being any undetected contamination exceeding the acceptance limits. Therefore, upon concurrence by NRC, the area may be released for unrestricted use.



## VII. REFERENCES

1. Special Nuclear Materials License No. SNM-21 and Technical Specification for Operations at Atomics Internatipnal, AI-75-46 and License Conditions
2. "Disposal or Onsite Storage of Thorium or Uranium Wastes from Past Operations," Federal Register 46, (205), 52061, October 23, 1981
3. "Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use," ("DECON-1") State of California, Radiologic Health Branch, Department of Health Services (June 1977)



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## APPENDIX A

### SURVEY RESULTS



Building 004 Not Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CPM	DEVI/100 CPM 30
01	E	0103	ALPHA	5	0
01	E	0503	ALPHA	17	18
01	S	0303	ALPHA	50	66
01	F	0303	ALPHA	32	41
01	F	0106	ALPHA	10	7
01	N	0301	ALPHA	6	1
01	W	0202	ALPHA	9	5
01	W	0602	ALPHA	3	-3
01	C	0206	ALPHA	3	-3
01	C	0201	ALPHA	12	11
01	N	0301	BETA	183	25
01	W	0202	BETA	174	-50
01	W	0602	BETA	196	133
01	C	0201	BETA	241	509
01	C	0206	BETA	210	250
01	F	0106	BETA	309	1057
01	F	0106	BETA	288	871
01	N	0301	BETA	183	25
01	W	0202	BETA	174	-50
01	W	0602	BETA	196	133
01	C	0206	BETA	210	250
01	C	0201	BETA	241	509
01	E	0103	BETA	191	77
01	E	0503	BETA	201	163
01	S	0303	BETA	203	223
01	F	0303	BETA	330	1274
01	F	0106	BETA	430	2135
02	N	0102	ALPHA	7	3
02	N	0303	ALPHA	8	4
02	N	0701	ALPHA	11	9
02	E	0203	ALPHA	9	6
02	E	0502	ALPHA	7	3
02	S	0102	ALPHA	6	1
02	S	0403	ALPHA	10	8
02	S	0702	ALPHA	3	-3
02	W	0202	ALPHA	9	6
02	W	0601	ALPHA	10	8
02	F	0105	ALPHA	11	9
02	F	0201	ALPHA	19	22
02	F	0504	ALPHA	28	36
02	F	0702	ALPHA	21	25
02	F	0706	ALPHA	23	28
02	C	0203	ALPHA	11	9
02	C	0206	ALPHA	6	1
02	C	0301	ALPHA	3	-3
02	C	0604	ALPHA	3	-3
02	C	0606	ALPHA	9	6
02	N	0102	BETA	195	111
02	N	0303	BETA	200	154



Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CPM	DEVI/100 CPM SD
02	N	0701	BETA	170	-34
02	E	0203	BETA	190	60
02	E	0502	BETA	213	265
02	S	0102	BETA	164	-154
02	S	0403	BETA	176	-51
02	S	0702	BETA	162	-171
02	W	0202	BETA	188	51
02	W	0601	BETA	177	-42
02	F	0105	BETA	349	1431
02	F	0201	BETA	311	1105
02	F	0504	BETA	357	1499
02	F	0702	BETA	369	1602
02	F	0706	BETA	307	1071
02	C	0203	BETA	219	317
02	C	0206	BETA	208	222
02	C	0301	BETA	190	68
02	C	0604	BETA	200	154
02	C	0606	BETA	191	77
02	F	0105	BETA	248	515
02	F	0702	BETA	274	746
03	N	0103	ALPHA	7	7
03	N	0502	ALPHA	2	0
03	N	0701	ALPHA	6	6
03	E	0302	ALPHA	6	6
03	E	0602	ALPHA	7	7
03	S	0103	ALPHA	3	1
03	S	0402	ALPHA	7	7
03	S	0703	ALPHA	4	3
03	W	0102	ALPHA	3	1
03	W	0503	ALPHA	3	1
03	C	0203	ALPHA	3	1
03	C	0402	ALPHA	5	4
03	C	0407	ALPHA	3	1
03	C	0601	ALPHA	3	1
03	C	0605	ALPHA	5	4
03	F	0104	ALPHA	14	17
03	F	0301	ALPHA	13	15
03	F	0307	ALPHA	22	30
03	F	0403	ALPHA	26	36
03	F	0606	ALPHA	6	4
03	N	0103	BETA	214	281
03	N	0502	BETA	181	8
03	N	0701	BETA	205	870
03	E	0302	BETA	179	-8
03	E	0602	BETA	155	-207
03	S	0103	BETA	207	223
03	S	0402	BETA	170	-02
03	S	0703	BETA	187	58
03	W	0102	BETA	212	265





Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS	CHI DR/100	CHI
					sq	
03	W	0503	BETA	213		273
03	C	0203	BETA	292		928
03	C	0402	BETA	196		157
03	C	0407	BETA	221		339
03	C	0601	BETA	233		439
03	C	0505	BETA	239		489
03	F	0104	BETA	327		1227
03	F	0301	BETA	388		1740
03	F	0307	BETA	292		933
03	F	0403	BETA	357		1480
03	F	0606	BETA	328		1236
04	W	0601	ALPHA	3		0
04	C	0202	ALPHA	1		-3
04	C	0206	ALPHA	5		3
04	W	0601	BETA	197		134
04	C	0202	BETA	220		327
04	C	0206	BETA	194		109
04	N	0103	ALPHA	13		16
04	E	0303	ALPHA	8		9
04	E	0602	ALPHA	5		4
04	S	0303	ALPHA	6		6
04	W	0303	ALPHA	5		4
04	F	0202	ALPHA	29		40
04	F	0503	ALPHA	229		344
04	N	0103	ALPHA	13		18
04	E	0303	ALPHA	8		9
04	E	0602	ALPHA	5		4
04	S	0303	ALPHA	6		6
04	W	0303	ALPHA	5		4
04	F	0202	ALPHA	29		44
04	F	0503	ALPHA	229		375
04	N	0103	BETA	200		177
04	E	0303	BETA	194		126
04	E	0602	BETA	159		-152
04	S	0303	BETA	226		366
04	W	0303	BETA	244		531
04	F	0202	BETA	352		1400
04	F	0503	BETA	313		1036
04	F	0503	ALPHA	4		1
04	F	0503	ALPHA	3		0
05	N	0102	ALPHA	4		1
05	N	0501	ALPHA	18		25
05	N	0803	ALPHA	11		13
05	N	1402	ALPHA	10		11
05	E	0401	ALPHA	6		5
05	S	0101	ALPHA	3		0
05	S	1201	ALPHA	11		13
05	W	0701	ALPHA	8		8
05	C	0202	ALPHA	6		5



Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CM	DEM/100 CM SQ
05	C	0205	ALPHA	3	0
05	C	0502	ALPHA	6	5
05	C	0507	ALPHA	8	8
05	C	0706	ALPHA	10	11
05	C	0801	ALPHA	7	6
05	C	1006	ALPHA	11	13
05	C	1207	ALPHA	12	15
05	S	1402	ALPHA	5	1
05	W	0402	ALPHA	4	0
05	F	0302	ALPHA	38	57
05	F	0105	ALPHA	36	54
05	F	0504	ALPHA	71	113
05	F	0601	ALPHA	128	209
05	C	1305	ALPHA	6	3
05	C	1102	ALPHA	8	6
05	C	1401	ALPHA	3	-1
05	N	1102	ALPHA	7	3
05	E	0202	ALPHA	3	-3
05	E	0703	ALPHA	7	3
05	S	0603	ALPHA	10	8
05	S	0903	ALPHA	11	9
05	W	0103	ALPHA	12	11
05	F	0307	ALPHA	43	68
05	F	0806	ALPHA	84	126
05	F	0903	ALPHA	43	60
05	F	1101	ALPHA	29	38
05	F	1204	ALPHA	38	52
05	F	1207	ALPHA	41	57
05	F	1402	ALPHA	22	27
05	N	1102	BETA	267	695
05	E	0202	BETA	196	114
05	E	0703	BETA	185	24
05	S	0603	BETA	183	0
05	S	0903	BETA	460	2274
05	W	0103	BETA	161	-171
05	F	0307	BETA	346	1341
05	F	0806	BETA	329	1202
05	F	0903	BETA	261	646
05	F	1101	BETA	320	1128
05	F	1204	BETA	339	1284
05	F	1207	BETA	333	1235
05	F	1402	BETA	314	1079
05	F	0601	ALPHA	4	1
05	S	0903	BETA	212	207
05	N	0102	BETA	169	-48
05	N	0501	BETA	175	0
05	N	0803	BETA	212	296
05	N	1402	BETA	328	1227
05	E	0401	BETA	180	40



Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CPM	DEVI/100 CPM SQ
05	S	0101	BETA	189	112
05	S	1201	BETA	210	280
05	W	0701	BETA	187	96
05	C	0202	BETA	197	176
05	C	0205	BETA	231	449
05	C	0602	BETA	210	280
05	C	0507	BETA	223	384
05	C	0706	BETA	210	280
05	C	0801	BETA	236	489
05	C	1006	BETA	188	104
05	C	1207	BETA	223	364
05	F	0504	BETA	333	1284
05	S	1402	BETA	193	167
05	W	0402	BETA	183	87
05	F	0302	BETA	372	1596
05	F	0105	BETA	345	1380
05	F	0601	BETA	319	1173
05	C	1305	BETA	232	478
05	C	1102	BETA	248	606
05	C	1401	BETA	283	885
06	E	0102	ALPHA	1	-6
06	E	0503	ALPHA	3	-3
06	S	0203	ALPHA	7	3
06	W	0202	ALPHA	4	-1
06	W	0503	ALPHA	3	-3
06	F	0106	ALPHA	21	24
06	F	0202	ALPHA	49	67
06	N	0101	ALPHA	9	6
06	C	0202	ALPHA	7	3
06	C	0206	ALPHA	2	-4
06	N	0101	BETA	212	267
06	C	0202	BETA	219	325
06	C	0206	BETA	211	258
06	E	0102	BETA	215	284
06	E	0503	BETA	286	895
06	S	0203	BETA	180	-17
06	W	0202	BETA	183	8
06	W	0503	BETA	183	8
06	F	0106	BETA	288	912
06	F	0202	BETA	369	1610
07	N	0602	ALPHA	9	6
07	S	2103	ALPHA	5	0
07	F	0401	ALPHA	24	30
07	F	0702	ALPHA	16	17
07	N	0602	BETA	157	-204
07	S	2103	BETA	211	237
07	F	0401	BETA	322	1145
07	F	0702	BETA	449	2184
07	C	0302	ALPHA	9	C



Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CM	DEVI/100 CM
07	F	1802	ALPHA	10	8
07	S	0603	ALPHA	12	11
07	S	1402	ALPHA	11	10
07	F	1101	ALPHA	44	65
07	F	1502	ALPHA	54	81
07	F	2001	ALPHA	9	6
07	N	1203	ALPHA	10	8
07	N	1203	BETA	159	-155
07	N	1802	BETA	174	-25
07	S	0603	BETA	160	-147
07	S	1402	BETA	161	-138
07	F	1101	BETA	290	978
07	F	1502	BETA	312	1169
07	F	2001	BETA	243	571
07	C	0302	BETA	215	329
07	N	2303	ALPHA	6	4
07	C	0801	ALPHA	6	4
07	C	1302	ALPHA	3	0
07	C	1701	ALPHA	3	0
07	C	2402	ALPHA	3	0
07	S	1702	ALPHA	2	-1
07	F	2502	ALPHA	10	10
07	N	2303	BETA	170	-107
07	C	0801	BETA	172	-89
07	C	1302	BETA	151	-276
07	C	1701	BETA	194	107
07	C	2402	BETA	158	-214
07	S	1702	BETA	165	-222
07	F	2502	BETA	258	604
07	F	0702	BETA	316	1120
07	F	0702	BETA	331	1253
07	F	0702	BETA	349	1413
07	F	0702	BETA	310	1054
08	N	0302	ALPHA	14	14
08	E	0302	ALPHA	9	6
08	E	0702	ALPHA	18	20
08	E	0902	ALPHA	15	16
08	S	0102	ALPHA	13	12
08	W	0202	ALPHA	12	11
08	W	0602	ALPHA	47	67
08	W	0902	ALPHA	5	0
08	F	0102	ALPHA	12	11
08	F	0104	ALPHA	11	9
08	F	0306	ALPHA	15	16
08	C	0103	ALPHA	4	-1
08	C	0105	ALPHA	3	-3
08	N	0102	ALPHA	2	-4
08	E	0202	ALPHA	5	0
08	E	0602	ALPHA	28	36



Building 004 Hot Chemistry Labs - Final Survey

ROOM	SUR	GRID	TYPE	GROSS CPN	DEP/100 C1 SQ
09	E	1202	ALPHA	6	1
09	S	0202	ALPHA	12	10
09	W	0101	ALPHA	4	-1
09	W	0302	ALPHA	14	13
09	W	0602	ALPHA	4	-1
09	F	0104	ALPHA	28	35
09	F	0201	ALPHA	11	9
09	F	0306	ALPHA	14	13
09	C	0103	ALPHA	6	1
09	C	0302	ALPHA	7	3
09	C	0306	ALPHA	6	1
09	E	0602	BETA	369	1591
09	E	0602	ALPHA	33	47
09	S	0202	BETA	209	232
09	W	0101	BETA	194	103
09	W	0302	BETA	179	-25
09	W	0602	BETA	200	154
09	F	0104	BETA	317	1152
09	F	0201	BETA	306	1067
09	F	0306	BETA	325	1231
09	C	0103	BETA	214	275
09	C	0302	BETA	232	430
09	C	0306	BETA	220	327



SHEAR DATA — 004 Hot Chem Lab

ROOM	SUR	GRID	TYPE	ALPHA COUNTS	BETA COUNTS	ALPHA DPM/100 SQ	BETA-GMIA DPM/100 SQ
01	E	0103	SHEAR	1	134	-0.7	4.6
01	F	0103	SHEAR	4	140	1.4	9.2
01	C	0201	SHEAR	3	127	0.7	-0.7
01	W	0202	SHEAR	2	128	0.0	0.0
01	C	0206	SHEAR	5	127	2.2	-0.7
01	N	0301	SHEAR	2	136	0.0	6.1
01	S	0303	SHEAR	13	126	8.2	-1.5
01	F	0303	SHEAR	5	140	2.2	9.2
01	E	0503	SHEAR	14	142	8.9	10.0
01	W	0502	SHEAR	1	120	-0.7	-6.1
02	N	0102	SHEAR	1	145	-0.7	10.8
02	S	0102	SHEAR	2	132	0.0	0.7
02	F	0105	SHEAR	4	112	1.4	-14.6
02	F	0201	SHEAR	1	147	-0.7	12.3
02	W	0202	SHEAR	5	142	2.1	8.4
02	E	0203	SHEAR	5	130	2.1	-0.7
02	C	0203	SHEAR	11	155	6.3	10.5
02	C	0206	SHEAR	5	144	2.1	10.0
02	C	0301	SHEAR	4	136	1.4	3.0
02	N	0303	SHEAR	4	141	1.4	7.7
02	S	0403	SHEAR	5	149	2.1	13.8
02	E	0502	SHEAR	2	135	0.0	3.0
02	F	0504	SHEAR	4	132	1.4	0.7
02	W	0501	SHEAR	1	126	-0.7	-3.0
02	C	0604	SHEAR	1	124	-0.7	-5.4
02	C	0606	SHEAR	10	131	5.6	0.0
02	W	0701	SHEAR	5	125	2.1	-4.6
02	S	0702	SHEAR	3	120	0.7	-8.4
02	F	0702	SHEAR	7	146	3.5	11.5
02	F	0706	SHEAR	7	140	3.5	6.9
03	W	0102	SHEAR	3	140	0.7	6.9
03	N	0103	SHEAR	3	131	0.7	0.0
03	S	0103	SHEAR	5	135	2.1	3.0
03	F	0104	SHEAR	2	129	0.0	-1.5
03	C	0203	SHEAR	4	123	1.4	-6.1
03	F	0301	SHEAR	1	131	-0.7	0.0
03	E	0302	SHEAR	3	148	0.7	13.1
03	F	0307	SHEAR	7	145	3.6	10.8
03	S	0402	SHEAR	11	131	6.5	0.0
03	C	0402	SHEAR	5	115	2.1	-12.3
03	F	0403	SHEAR	7	131	3.6	0.0
03	C	0407	SHEAR	5	138	2.1	5.4
03	N	0502	SHEAR	7	134	3.6	2.3
03	W	0503	SHEAR	3	137	0.7	4.6
03	C	0601	SHEAR	2	118	0.0	-10.0
03	E	0602	SHEAR	0	138	-1.4	5.4
03	C	0605	SHEAR	1	119	-0.7	-9.2
03	F	0606	SHEAR	2	123	0.0	-6.1



SMEAR DATA — 004 Hot Chem Lab

ROOM	SUR	GRID	TYPE	ALPHA COUNTS	BETA COUNTS	ALPHA DPM/100 SQ	BETA-GAMMA DPM/100 SQ
03	N	0701	SMEAR	3	136	0.7	3.8
03	S	0703	SMEAR	4	137	1.4	4.6
04	N	0103	SMEAR	18	142	12.0	15.4
04	F	0202	SMEAR	5	133	2.2	8.4
04	C	0202	SMEAR	2	136	0.0	10.8
04	C	0206	SMEAR	12	130	7.5	6.1
04	E	0303	SMEAR	14	210	9.0	67.9
04	S	0303	SMEAR	6	141	3.0	14.6
04	W	0303	SMEAR	4	150	1.5	21.6
04	F	0503	SMEAR	6	138	3.0	12.3
04	W	0601	SMEAR	5	143	2.2	16.2
04	E	0602	SMEAR	5	119	2.2	-2.3
05	S	0101	SMEAR	3	135	0.7	12.4
05	N	0102	SMEAR	6	134	3.0	11.6
05	W	0103	SMEAR	4	143	1.5	18.6
05	F	0105	SMEAR	3	131	0.7	9.3
05	E	0202	SMEAR	2	127	0.0	6.2
05	C	0202	SMEAR	5	125	2.2	4.6
05	C	0205	SMEAR	5	102	2.2	-13.2
05	F	0302	SMEAR	3	117	0.7	-1.5
05	F	0307	SMEAR	3	117	0.7	-1.5
05	E	0401	SMEAR	2	131	0.0	9.3
05	W	0402	SMEAR	9	107	5.2	-9.3
05	N	0501	SMEAR	6	116	3.0	-2.3
05	F	0504	SMEAR	4	146	1.5	21.0
05	C	0507	SMEAR	4	132	1.5	10.1
05	F	0601	SMEAR	4	132	1.5	10.1
05	C	0602	SMEAR	2	112	0.0	-5.4
05	S	0603	SMEAR	14	134	9.0	11.6
05	W	0701	SMEAR	8	124	4.5	3.8
05	E	0703	SMEAR	3	117	0.7	-1.5
05	C	0706	SMEAR	4	123	1.5	3.1
05	C	0801	SMEAR	1	132	-0.7	10.1
05	N	0803	SMEAR	3	131	0.7	9.3
05	F	0806	SMEAR	8	156	4.5	20.7
05	S	0903	SMEAR	9	148	5.2	22.5
05	F	0903	SMEAR	0	119	-1.5	0.0
05	C	1006	SMEAR	2	104	0.0	-11.6
05	F	1101	SMEAR	6	144	3.0	19.4
05	N	1102	SMEAR	2	155	0.0	28.0
05	C	1102	SMEAR	3	142	0.7	17.8
05	S	1201	SMEAR	7	120	3.7	0.7
05	F	1204	SMEAR	4	136	1.5	13.2
05	F	1207	SMEAR	5	123	2.2	3.1
05	C	1207	SMEAR	4	144	1.5	19.4
05	C	1305	SMEAR	2	126	0.0	5.4
05	C	1401	SMEAR	1	129	-0.7	7.7
05	N	1402	SMEAR	0	127	-1.5	6.2



SHEAR DATA — 004 Hot Chem Lab

ROOM	SUR	GRID	TYPE	ALPHA COUNTS	BETA COUNTS	ALPHA DPM/100 SQ	BETA-GAMMA DPM/100 CM SQ
05	S	1402	SHEAR	5	127	3.0	6.2
05	F	1402	SHEAR	3	142	0.7	17.8
06	N	0101	SHEAR	1	134	-7	4.6
06	E	0102	SHEAR	4	140	1.4	9.2
06	F	0106	SHEAR	5	124	2.2	-3.0
06	W	0202	SHEAR	2	109	0.0	-14.7
06	F	0202	SHEAR	3	146	4.4	13.9
06	C	0202	SHEAR	0	125	-1.4	-2.3
06	S	0203	SHEAR	2	137	0.0	6.9
06	C	0206	SHEAR	1	128	-7	0.0
06	E	0503	SHEAR	3	150	0.7	17.0
06	W	0503	SHEAR	2	130	0.0	1.5
07	C	0302	SHEAR	4	132	1.5	-4.6
07	F	0401	SHEAR	5	123	2.2	-11.5
07	N	0602	SHEAR	2	208	0.0	59.7
07	S	0603	SHEAR	4	132	1.5	1.5
07	F	0702	SHEAR	3	128	0.7	-7.6
07	C	0801	SHEAR	5	114	2.2	-18.4
07	F	1101	SHEAR	6	120	3.0	-13.8
07	N	1203	SHEAR	3	240	0.7	64.2
07	C	1302	SHEAR	6	133	3.0	-3.8
07	S	1402	SHEAR	3	128	0.7	-1.5
07	F	1502	SHEAR	5	114	2.2	-18.4
07	C	1701	SHEAR	6	131	3.0	-5.3
07	S	1702	SHEAR	4	139	1.5	6.8
07	N	1802	SHEAR	1	129	-7	-7
07	F	2001	SHEAR	2	150	0.0	9.2
07	S	2103	SHEAR	3	117	4.5	-9.9
07	N	2303	SHEAR	6	130	3.0	0.0
07	C	2402	SHEAR	1	133	-7	-3.8
07	F	2502	SHEAR	3	143	4.5	3.8
08	S	0102	SHEAR	3	136	0.7	0.0
08	F	0102	SHEAR	3	129	0.7	2.4
08	C	0103	SHEAR	1	127	-7	0.8
08	F	0104	SHEAR	2	111	0.0	-12.0
08	C	0105	SHEAR	4	139	1.5	10.4
08	W	0202	SHEAR	5	147	2.3	16.8
08	N	0302	SHEAR	5	127	2.3	0.8
08	E	0302	SHEAR	4	126	1.5	0.0
08	F	0306	SHEAR	5	131	2.3	4.0
08	W	0602	SHEAR	3	106	0.7	-15.0
08	E	0702	SHEAR	10	159	6.2	26.4
08	E	0902	SHEAR	4	143	1.5	13.8
08	W	0902	SHEAR	2	140	0.0	11.2
08	N	0101	SHEAR	4	114	1.5	-7.2
08	N	0102	SHEAR	5	141	2.3	12.0
08	C	0103	SHEAR	2	124	0.0	0.8
08	F	0104	SHEAR	4	135	1.5	9.6





SHEAR DATA — 004 Not Chem Lab

ROOM	SUR	GRID	TYPE	ALPHA COUNTS	BETA COUNTS	ALPHA DPM/100 SQ	BETA-GM/A DPM/100 CM
09	F	0201	SHEAR	3	153	0.7	24.0
09	E	0202	SHEAR	2	127	0.0	0.8
09	S	0202	SHEAR	3	137	0.7	11.2
09	W	0302	SHEAR	6	136	3.0	10.4
09	C	0302	SHEAR	3	124	0.7	0.8
09	F	0306	SHEAR	0	120	-1.5	-2.4
09	C	0306	SHEAR	1	135	-.7	9.6
09	E	0602	SHEAR	4	141	1.5	12.0
09	W	0602	SHEAR	0	130	-1.5	5.6
09	E	0602	SHEAR	5	144	2.2	4.6
09	E	1202	SHEAR	4	137	1.5	8.3



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## APPENDIX B

ANNEX C TO SPECIAL NUCLEAR MATERIAL  
LICENSE NO. SNM-21

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ANNEX C

GUIDELINES FOR DECONTAMINATION OF FACILITIES AND EQUIPMENT  
PRIOR TO RELEASE FOR UNRESTRICTED USE  
OR TERMINATION OF LICENSES FOR BYPRODUCT, SOURCE,  
OR SPECIAL NUCLEAR MATERIAL

U. S. Nuclear Regulatory Commission  
Division of Fuel Cycle and  
Material Safety  
Washington, D.C. 20555

NOVEMBER 1976

SEP 1 1977

The instructions in this guide in conjunction with Table I specify the radioactivity and radiation exposure rate limits which should be used in accomplishing the decontamination and survey of surfaces or premises and equipment prior to abandonment or release for unrestricted use. The limits in Table I do not apply to premises, equipment, or scrap containing induced radioactivity for which the radiological considerations pertinent to their use may be different. The release of such facilities or items from regulatory control will be considered on a case-by-case basis.

1. The licensee shall make a reasonable effort to eliminate residual contamination.
2. Radioactivity on equipment or surfaces shall not be covered by paint, plating, or other covering material unless contamination levels, as determined by a survey and documented, are below the limits specified in Table I prior to applying the covering. A reasonable effort must be made to minimize the contamination prior to use of any covering.
3. The radioactivity on the interior surfaces of pipes, drain lines, or ductwork shall be determined by making measurements at all traps, and other appropriate access points, provided that contamination at these locations is likely to be representative of contamination on the interior of the pipes, drain lines, or ductwork. Surfaces of premises, equipment, or scrap which are likely to be contaminated but are of such size, construction, or location as to make the surface inaccessible for purposes of measurement shall be presumed to be contaminated in excess of the limits.
4. Upon request, the Commission may authorize a licensee to relinquish possession or control of premises, equipment, or scrap having surfaces contaminated with materials in excess of the limits specified. This may include, but would not be limited to, special circumstances such as razing of buildings, transfer of premises to another organization continuing work with radioactive materials, or conversion of facilities to a long-term storage or standby status. Such requests must:
  - a. Provide detailed, specific information describing the premises, equipment or scrap, radioactive contaminants, and the nature, extent, and degree of residual surface contamination.
  - b. Provide a detailed health and safety analysis which reflects that the residual amounts of materials on surface areas, together with other considerations such as prospective use of the premises, equipment or scrap, are unlikely to result in an unreasonable risk to the health and safety of the public.

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5. Prior to release of premises for unrestricted use, the licensee shall make a comprehensive radiation survey which establishes that contamination is within the limits specified in Table I. A copy of the survey report shall be filed with the Division of Fuel Cycle and Material Safety, USNRC, Washington, D.C. 20555, and also the Director of the Regional Office of the Office of Inspection and Enforcement, USNRC, having jurisdiction. The report should be filed at least 30 days prior to the planned date of abandonment. The survey report shall:

- a. Identify the premises.
- b. Show that reasonable effort has been made to eliminate residual contamination.
- c. Describe the scope of the survey and general procedures followed.
- d. State the findings of the survey in units specified in the instruction.

Following review of the report, the NRC will consider visiting the facilities to confirm the survey.

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TABLE 1

## ACCEPTABLE SURFACE CONTAMINATION LEVELS

NUCLIDES <sup>a</sup>	AVERAGE <sup>b c f</sup>	MAXIMUM <sup>b d f</sup>	REMOVABLE <sup>b e f</sup>
U-nat, U-235, U-238, and associated decay products	5,000 dpm $\alpha$ /100 cm <sup>2</sup>	15,000 dpm $\alpha$ /100 cm <sup>2</sup>	1,000 dpm $\alpha$ /100 cm <sup>2</sup>
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100 dpm/100 cm <sup>2</sup>	300 dpm/100 cm <sup>2</sup>	20 dpm/100 cm <sup>2</sup>
Th-nat, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000 dpm/100 cm <sup>2</sup>	3,000 dpm/100 cm <sup>2</sup>	200 dpm/100 cm <sup>2</sup>
Beta-gamma emitters (nuclides with decay modes other than alpha emission or spontaneous fission) except SR-90 and others noted above.	5,000 dpm $\beta\gamma$ /100 cm <sup>2</sup>	15,000 dpm $\beta\gamma$ /100 cm <sup>2</sup>	1,000 dpm $\beta\gamma$ /100 cm <sup>2</sup>

<sup>a</sup>Where surface contamination by both alpha- and beta-gamma-emitting nuclides exists, the limits established for alpha- and beta-gamma-emitting nuclides should apply independently.

<sup>b</sup>As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

<sup>c</sup>Measurements of average contaminant should not be averaged over more than 1 square meter. For objects of less surface area, the average should be derived for each such object.

<sup>d</sup>The maximum contamination level applies to an area of not more than 100 cm<sup>2</sup>.

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TABLE I  
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<sup>e</sup>The amount of removable radioactive material per 100 cm<sup>2</sup> of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionally and the entire surface should be wiped.

<sup>f</sup>The average and maximum radiation levels associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/hr at 1 cm and 1.0 mrad/hr at 1 cm, respectively, measured through not more than 7 milligrams per square centimeter of total absorber.

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